

October 29, 2004

Mr. James F. Mallay
Director, Regulatory Affairs
Framatome ANP
3815 Old Forest Road
Lynchburg, VA 24501

SUBJECT: DRAFT SAFETY EVALUATION FOR FRAMATOME ANP TOPICAL REPORT
BAW-10179(P), REVISION 5, "SAFETY CRITERIA AND METHODOLOGY FOR
ACCEPTABLE CYCLE RELOAD ANALYSES" (TAC NO. MB9684)

Dear Mr. Mallay:

By letter dated December 18, 2002, and its supplement dated March 5, 2004, Framatome ANP (FANP) submitted Topical Report (TR) BAW-10179(P), Revision 5, "Safety Criteria and Methodology for Acceptable Cycle Reload Analyses," to the staff for review. Enclosed for FANP's review and comment is a copy of the staff's draft safety evaluation (SE) for TR BAW-10179(P), Revision 5.

Pursuant to 10 CFR 2.390, we have determined that the enclosed draft SE does not contain proprietary information. However, we will delay placing the draft SE in the public document room for a period of ten working days from the date of this letter to provide you with the opportunity to comment on the proprietary aspects. If you believe that any information in the enclosure is proprietary, please identify such information line-by-line and define the basis pursuant to the criteria of 10 CFR 2.390. After ten working days, the draft SE will be made publicly available, and an additional ten working days are provided to you to comment on any factual errors or clarity concerns contained in the SE. The final SE will be issued after making any necessary changes and will be made publicly available. The staff's disposition of your comments on the draft SE will be discussed in the final SE.

To facilitate the staff's review of your comments, please provide a marked-up copy of the draft SE showing proposed changes and provide a summary table of the proposed changes.

In the event of any comments or questions, please contact Michelle Honcharik at (301) 415-1774.

Sincerely,

/RA/

Robert A. Gramm, Chief, Section 2
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Project No. 728

Enclosure: Draft Safety Evaluation

October 29, 2004

Mr. James F. Mallay
Director, Regulatory Affairs
Framatome ANP
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DRAFT SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
BAW-10179(P), REVISION 5, "SAFETY CRITERIA AND METHODOLOGY FOR
ACCEPTABLE CYCLE RELOAD ANALYSES"
FRAMATOME ANP
PROJECT NO. 728

1.0 INTRODUCTION

By letter dated December 18, 2002 (ADAMS Accession No. ML023600285), as supplemented by letter dated March 5, 2004 (ADAMS Accession No. ML040690723), Framatome ANP (FANP) submitted Topical Report (TR) BAW-10179(P), Revision 5, "Safety Criteria and Methodology for Acceptable Cycle Reload Analysis" for NRC staff review. BAW-10179(P) provides the criteria and methodology for determining cycle-specific limits and setpoints that are included in the Core Operating Limits Reports (COLRs) for the Babcock and Wilcox (B&W) 177-fuel assembly class of nuclear power plants. The TR serves as the approved methodology that is referenced in the COLR portion of the Administrative Controls section of a plant's technical specifications.

2.0 REGULATORY EVALUATION

In Revision 5 to BAW-10179(P), FANP added three TRs that have been approved by the NRC since Revision 4 of BAW-10179(P) was approved. Each TR is briefly described in an appendix, and the description includes the conditions and limitations on the applicability of the TR.

The following have been incorporated in BAW-10179P, Revision 5:

Appendix U: BAW 10164P-A, "RELAP5/MOD2 - B&W - An Advanced Computer Program for Light Water Reactor LOCA and Non-LOCA Transient Analysis," Revision 4, November 2002.

Appendix V: BAW-10241P-A, "BHTP DNB Correlation Applied With LYNXT," December 2002.

Appendix W: BAW-10166P-A, "BEACH - Best Estimate Analysis Core Heat Transfer, A Computer Program for Reflood Heat Transfer During LOCA," Revision 5, November 2002.

The NRC staff reviewed the safety evaluations that were issued for the above TRs, FANP's descriptions of the TRs, and the limitations, if any, on the applicability to B&W 177-fuel assembly class of nuclear power plants. The NRC staff finds that the summaries for the TRs,

including limitations on applicability to the B&W 177-fuel assembly class of nuclear power plants, are adequate.

3.0 CONCLUSION

The NRC staff concludes that BAW-10179(P), Revision 5, adds only TRs that are approved for B&W 177-fuel assembly class of nuclear power plants, and that the summaries of the TRs are adequate. Therefore, Revision 5 may be referenced in a plant's technical specifications as the current approved version of BAW-10179(P).

Principal Contributor: M. Honcharik

Date: October 29, 2004